

UKAEA

# Inventory predictions with FISPACT-II

## Charged particle irradiation

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ROYCE TRAINING: ION BEAM IRRADIATION AND  
CHARACTERISATION - BEST PRACTICE

March 2, 2023

## 1. Inventory simulations

- ▶ Needs & numerical methodology
- ▶ inputs & outputs
- ▶ neutron examples

## 2. Charged particle irradiation

- ▶  $p/\alpha$  differences from neutrons
- ▶ simulation methods
- ▶ proton monoenergetic examples

# 1: Introduction to inventory simulations

## What are inventory simulations?

- Inventory simulations are used to predict how the chemical composition of a material will be altered under (neutron/proton/ $\alpha$ ) irradiation
  - ▶ and/or by the decay of radioactive species
- other terms: transmutation, burn-up

## Motivation

- vital to have reliable predictions of these time-dependent changes
  - ▶ for inclusion in engineering design studies of nuclear systems, including reactors, accelerators, and ion-beams
  - ▶ for use to define operational limits & safe working lifetimes of components in fusion/fission devices
  - ▶ to evaluate expected masses and costs of waste disposal
  - ▶ to support the design of irradiation experiments including predictions of radiological hazards

## How to predict inventory change

- via solutions to Inventory rate equations:

$$\frac{dN_i}{dt} = \underbrace{-N_i(\lambda_i + \sigma_i\phi)}_{\text{loss}} + \sum_{j \neq i} \underbrace{N_j(\lambda_{ji} + \sigma_{ji}\phi)}_{\text{creation}}$$

- coupled differential equations
  - ▶ one equation for each nuclide  $i$  at concentration  $N_i$
  - ▶ typically solved numerically & used to evolve composition in time
- $\sigma_{ji}$  [ $\text{cm}^2$ ]: average reaction rate cross section for  $j \rightarrow i$  reaction
  - ▶ reactions include  $(n,\gamma)$ ,  $(n,\alpha)$ ,  $(n,2n)$ ,  $(p,n)$ ,  $(\alpha,n)$ , etc.
  - ▶ obtained by collapsing energy-dependent cross sections (typically from nuclear data libraries) with projectile  $(n/p/\alpha)$  energy spectra
  - ▶  $\sigma_i$  is sum over all  $i \rightarrow j$  reactions
- $\lambda_i, \lambda_{ji}$  decay constants [ $\text{s}^{-1}$ ]
- $\phi$  total projectile flux [ $\text{cm}^{-2} \text{s}^{-1}$ ]

## Methodology details

- assumed that irradiation spectrum (energy profile) is not altered by change in composition in a time-step
  - and also that spectrum is the same for all atoms in a material (zero dimensional approximation)
- equations are linear and coefficients are constant and independent of  $N$ s

∴ can be expressed in matrix form:

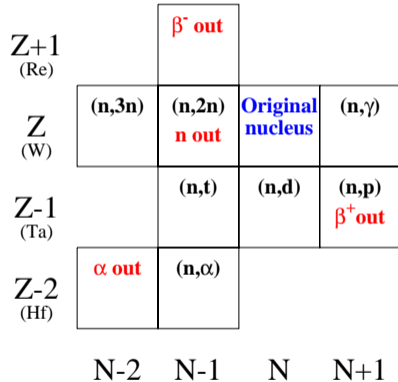
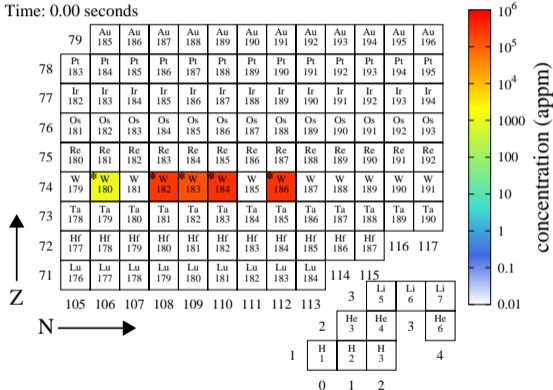
$$\dot{\mathbf{N}} = \mathbf{A}\mathbf{N} = (\mathbf{\Lambda} + \phi\mathbf{\Sigma})\mathbf{N}$$

- the matrix  $\mathbf{A}$  is sparse ( $\sim 3\%$  occupied even when fission is included)
- & stiff due to very large range of decay constants (i.e. small change in solutions have large fluctuations)
- the FISPACT-II inventory code uses the well-established LSODE package for this stiff and sparse system of linear ODEs

# Inventory evolution example

- 5-year irradiation of pure tungsten in D-T fusion neutron environment
- nuclide/isotope distribution changes with time

Time: 0.00 seconds



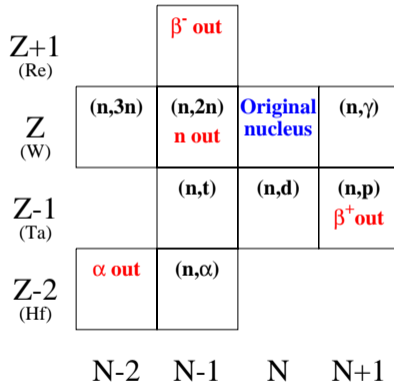
“chart of nuclide visualization”: *Nucl. Sci. Eng* **117** (2014) 291-306

appm - atomic parts per million  
 \* nuclide present in input composition  
 m - concentration dominated by metastable state



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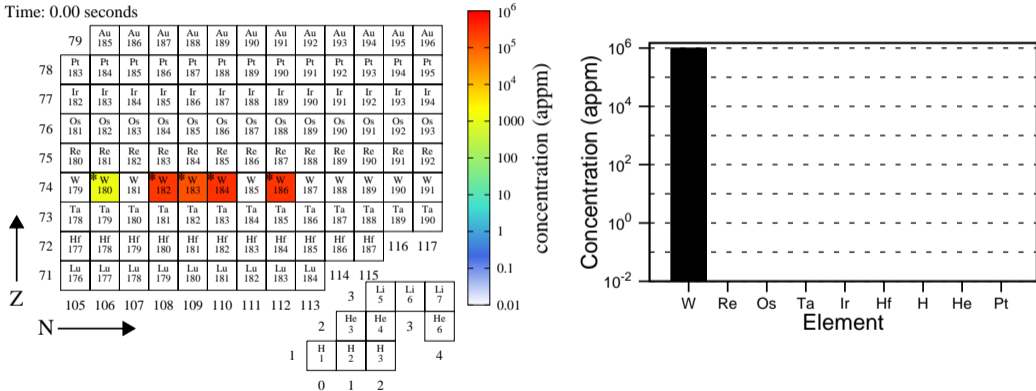


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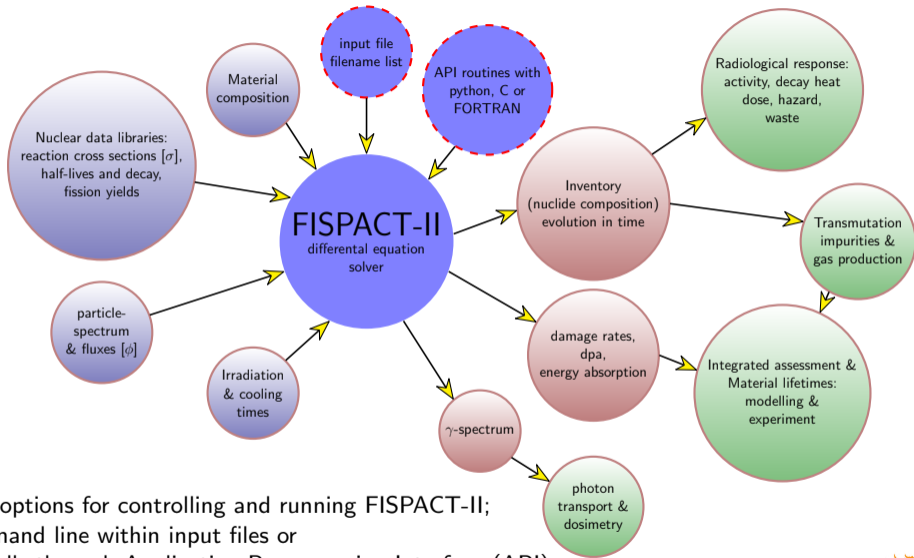
- multiphysics platform for predicting the composition changes under both neutron and charged-particle interactions
  - ▶ calculates activation, transmutation, burn-up, dpa, gas production, gamma spectra, etc.
- can use all major nuclear data libraries containing nuclear reaction data, radioactive decay data and fission yield data

[fispact.ukaea.uk](http://fispact.ukaea.uk)

[fispact.ukaea.uk/wiki](http://fispact.ukaea.uk/wiki)

- Available from OECD/NEA-databank  
<https://www.oecd-nea.org/tools/abstract/detail/nea-1890/>

# Inventory calculations: data flow



Two options for controlling and running FISPACT-II;  
command line within input files or  
via calls through Application Programming Interface (API)

## Types of radiological output

- Activity measured in becquerels (Bq) – number of disintegrations (decays) per second – the primary measure
  - ▶ can be separated by decay type –  $\alpha$ ,  $\beta$ ,  $\gamma$
- decay heat, measured in kilowatts (kW)
  - ▶ can be separated by decay type -  $\alpha$ ,  $\beta$ ,  $\gamma$
  - ▶ how much heat will be generated by the decay of radioactive species even after irradiation
- $\gamma$  dose rate, measured in Sieverts (Sv) per hour
  - ▶ contact or point dose approximations only in inventory codes
  - ▶  $\text{J kg}^{-1}$  deposition rate of radiation energy in biological tissue
  - ▶ and also ingestion and inhalation hazard conversions
- clearance index
  - ▶ IAEA based measure
  - ▶ a nuclide can be disposed of as if it were non radioactive when the index is less than 1





# Radiological response evolution example

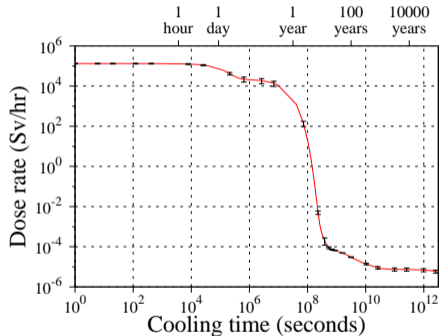
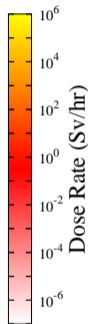
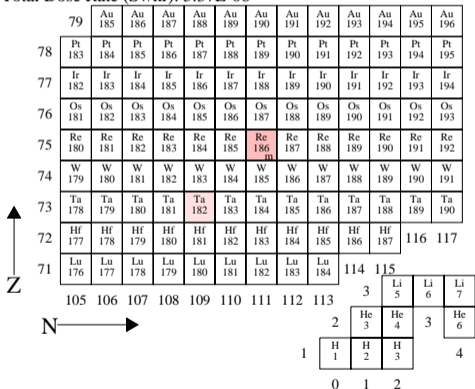
- Starting from contact dose rate after 5-year irradiation of pure W
- nuclide chart shows decay of radionuclides
- sum over all nuclides give total dose output

m – Dose Rate dominated by metastable nuclide(s)

# Radiological response evolution example

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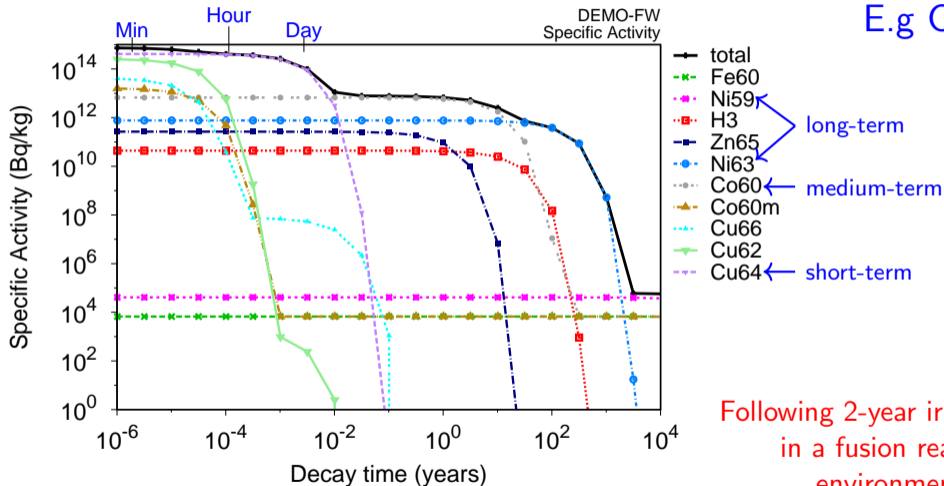
Time: 1.11E+05 years (cooling)  
 Total Dose Rate (Sv/hr): 5.37E-06



m – Dose Rate dominated by metastable nuclide(s)

# Activation response – full nuclide contributions

- Evolution for (dominant) nuclide contributions as a function of time



E.g Copper

Following 2-year irradiations  
in a fusion reactor  
environment

- shows which nuclides are important and when

## TENDL (latest version 2021<sup>†</sup>)

- TALYS-based Evaluated Nuclear Data Libraries
- generated using various physical, theoretical, and semi-empirical models
- fully-automated production with *near complete* coverage of nuclides & reactions
  - ▶ avoids under-estimation due to missing data
- contains data more than 2800 target nuclides with half-lives  $> 1$  second
  - ▶ includes sub-libraries for neutron, d, t, p,  $\alpha$ ,  $^3\text{He}$ ,  $\gamma$  exposure
  - ▶ need to be combined with a decay library, various available
    - decay2020, available from FISPACT website, is recommended
- processed versions for FISPACT-II are not pointwise – data is stored on a fixed energy grid; neutron one is higher resolution than charged particle ones

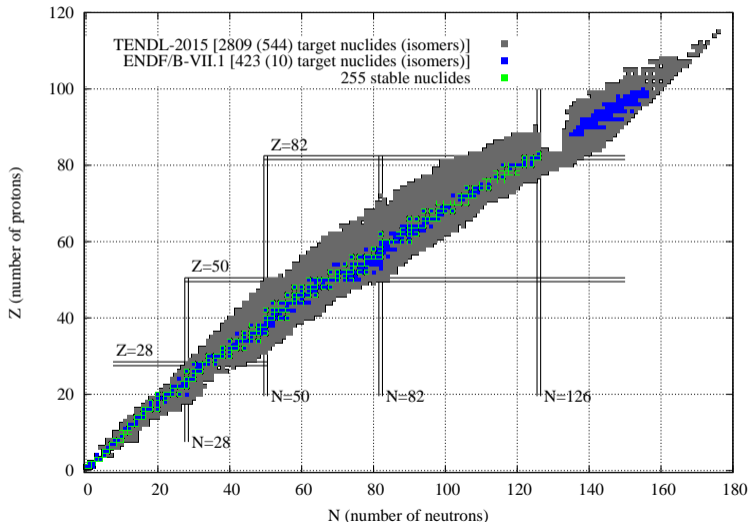
<sup>†</sup>A. J. Koning, D. Rochman, *et al.*. Release date: December 30, 2021.

[https://tendl.web.psi.ch/tendl\\_2021/tendl2021.html](https://tendl.web.psi.ch/tendl_2021/tendl2021.html)

FISPACT-II compatible versions available there

# TENDL nuclide coverage

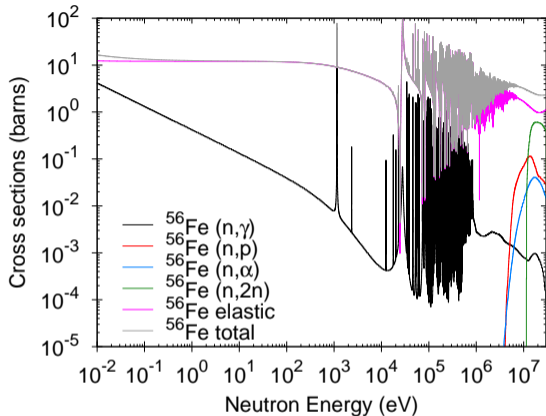
- Target nuclide coverage in TENDL libraries is more complete than elsewhere:



- Many more isomeric states are included as both targets (parents) and daughters of reactions – vital for correct prediction of activity

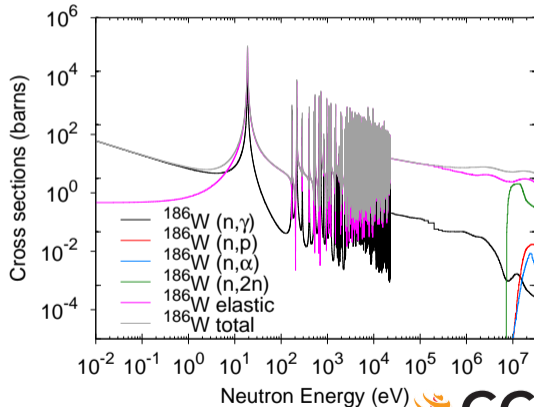
## 2. Charged particle irradiation

# Typical cross sections $\sigma$ – neutrons

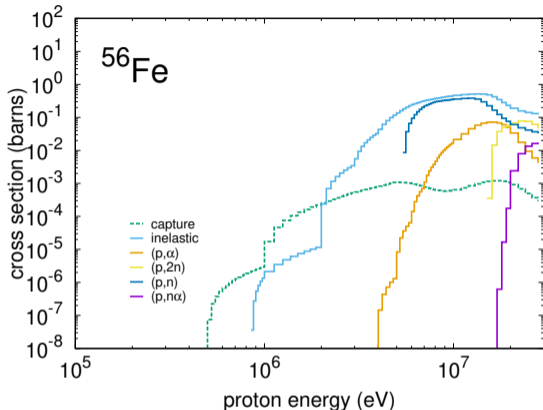


- Most channels are threshold type and are only 'open' at high energy
- but neutron capture can happen at all energies

- cross section (xs)  
 $\approx$  'reaction likelihood'
- Many different reactions possible on each nuclide/isotope

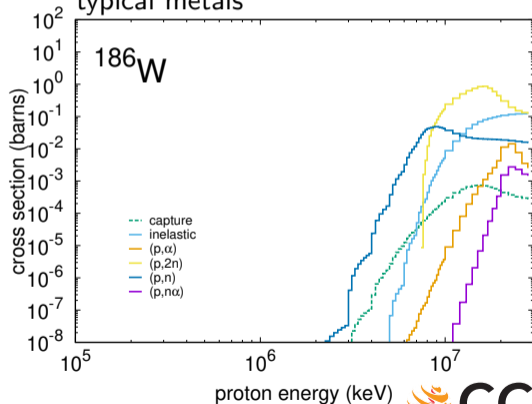


# Typical cross sections $\sigma$ – protons



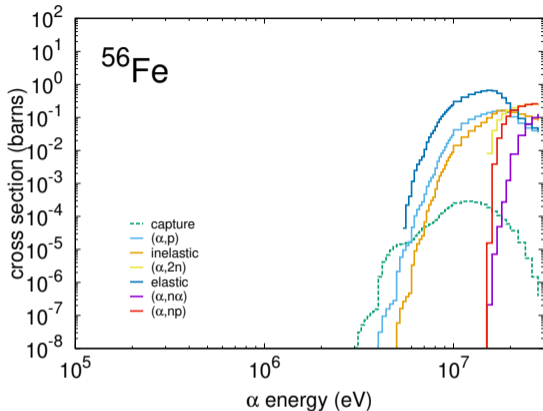
- similar break-up reactions as seen for neutron irradiation also occur

- under proton irradiation, capture has a threshold due to the coulomb barrier and only becomes significant at MeV energies in typical metals

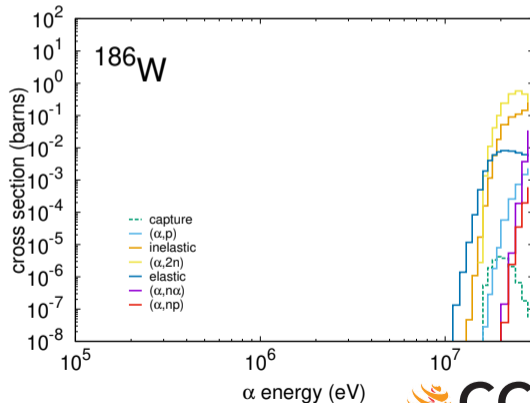




# Typical cross sections $\sigma - \alpha$



- similar under  $\alpha$  bombardment
- only high energy irradiation will cause transmutation and activation

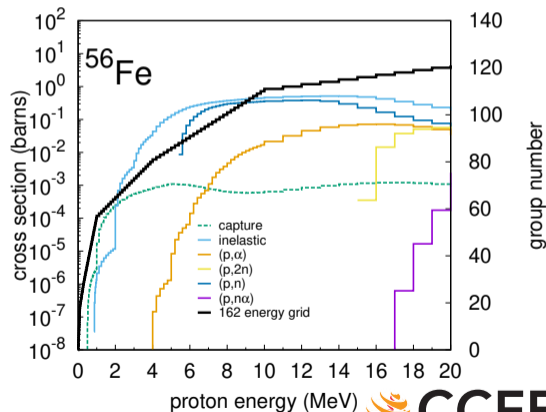


# Monoenergetic irradiation comparisons

- scoping of response as a function of beam (irradiation) energy

## Methodology

- monoenergetic irradiation beam approximated as particles in one or more of the energy bins used to store the cross section data
- Irradiation conditions:  
Flux of  $10^{13}$  particles  $\text{cm}^{-2} \text{s}^{-1}$   
6 month irradiation
- Activity calculated at end of irradiation and following 6 months of decay cooling
- identical simulations performed with neutrons, protons,  $\alpha$



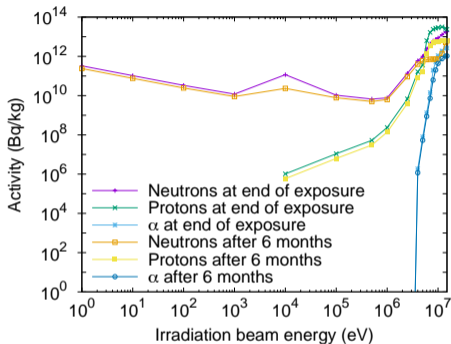
## Difference between particle types

- select particle type (`PROJECTILE` keyword or )
- point to the correct library of cross sections (from TENDL)
  - ▶ `xs_endf` in 'files'
- everything else the same

## Creating different beam energies

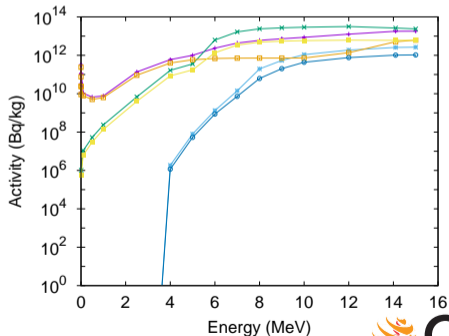
- create a pseudo-monoenergetic spectrum using `GRPCONVERT`
  - ▶ arbitrary flux file (`arb_flux`) specifying one energy bin that can be translated to a spectrum with the correct energy grid to merge with reaction data
- see wiki or manual for more details (& API manual for equivalent keys)

# monoenergetic beam results: Fe



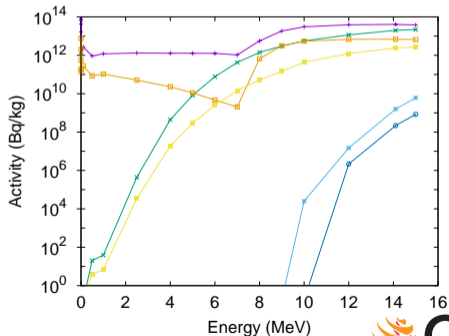
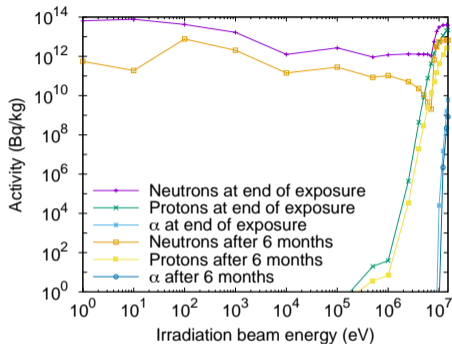
- Proton irradiation at low energy produces much less activation than equivalent energies of neutron irradiation
- however, above  $\sim 5$  MeV the p-induced activity can be higher than with neutrons

- Similar with  $\alpha$ -particle irradiation, although no scenario in this case where  $\alpha$ -induced activity is higher than the neutron case (at  $t=0$ )



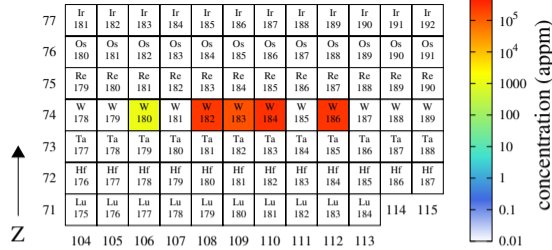
# monoenergetic beam results: W

- more extreme difference in some materials
- in W, the high capture cross sections at low neutron energies produces a high activation response

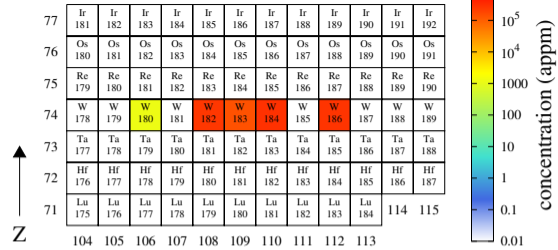


Protons vs neutrons  $W$   
at 14 MeV

Time: 0.00 seconds



Time: 0.00 seconds

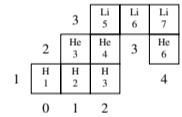
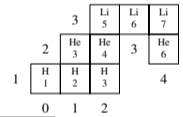


Z ↑

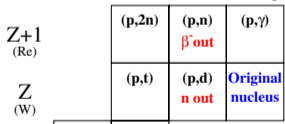
Z ↑

N →

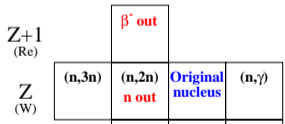
N →



W



Proton



neutron

N-3 N-2 N-1 N

N-2 N-1 N N+1

14 MeV beam

flux:  
10<sup>14</sup> particles  
cm<sup>-2</sup> s<sup>-1</sup>

W

Z+1 (Re)	(p,2n)	(p,n) $\beta^-$ out	(p, $\gamma$ )
Z (W)	(p,t)	(p,d) n out	Original nucleus
Z-1 (Ta)	(p,n $\alpha$ )	(p, $\alpha$ )	
	N-3	N-2	N-1

Proton

14 MeV  
beam

Z+1 (Re)	$\beta^-$ out		
Z (W)	(n,3n)	(n,2n) n out	Original nucleus
Z-1 (Ta)		(n,t)	(n,d)
Z-2 (Hf)	$\alpha$ out	(n, $\alpha$ )	(n,p) $\beta^+$ out
	N-2	N-1	N

neutron

flux:  
 $10^{14}$  particles  
 $\text{cm}^{-2} \text{s}^{-1}$



W

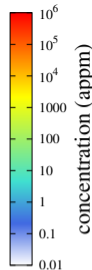
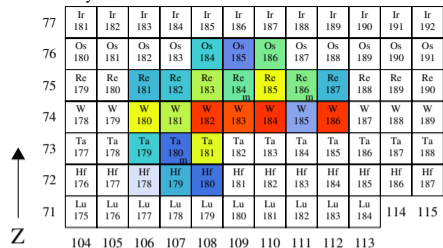
Proton

neutron

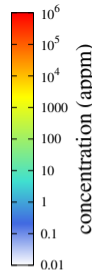
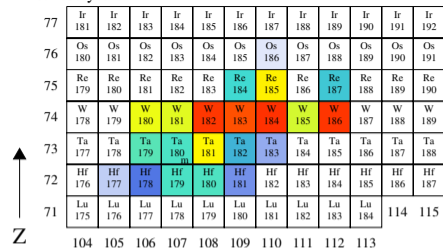
14 MeV  
beam

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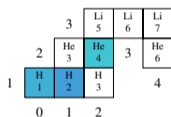
Time: 2.00 years



Time: 2.00 years

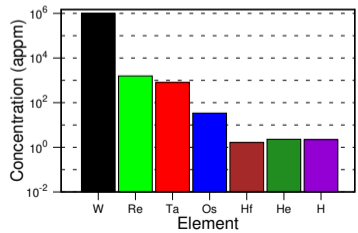
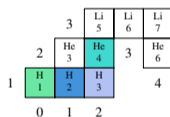


N →



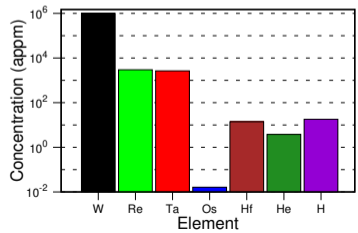
W

N →



Proton

14 MeV beam

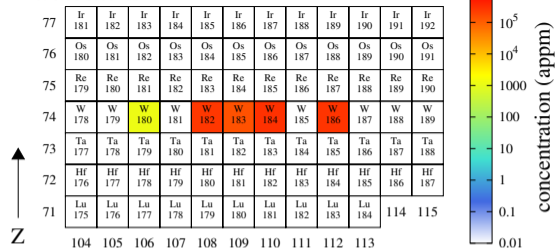


neutron

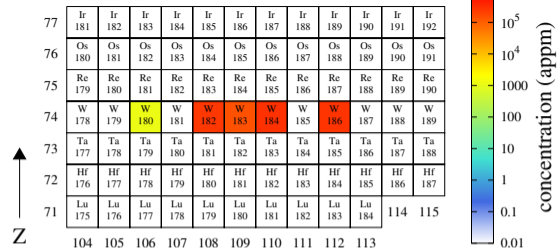
flux:  
 $10^{14}$  particles  
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$\alpha$  vs neutrons  
W at 14 MeV

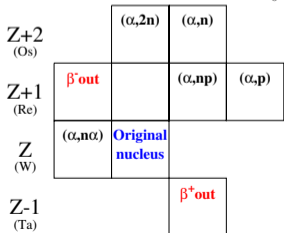
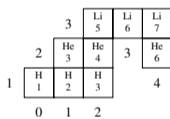
Time: 0.00 seconds



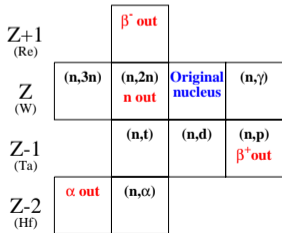
Time: 0.00 seconds



W



$\alpha$



neutron

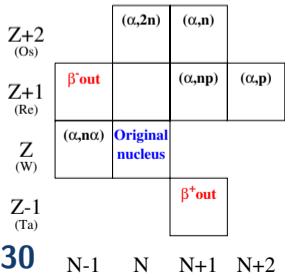
14 MeV beam

flux:  
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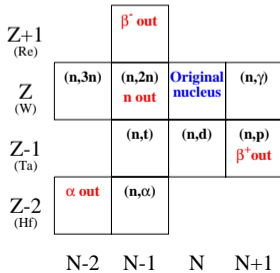
W

$\alpha$

neutron



14 MeV beam



flux:  
 $10^{14}$  particles  
 $\text{cm}^{-2} \text{s}^{-1}$

$\alpha$

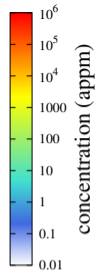
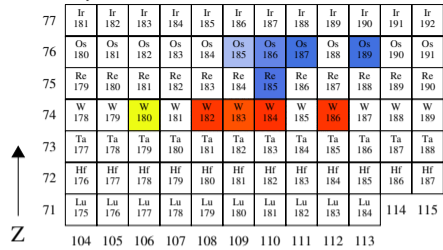
W

neutron

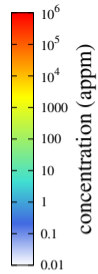
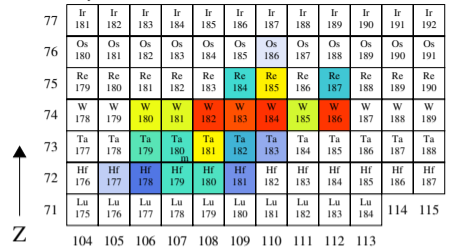
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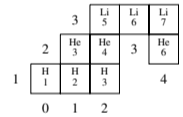


Z ↑

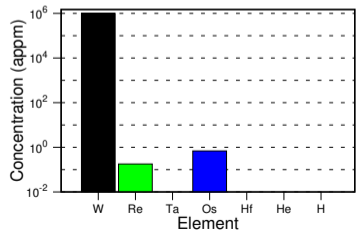
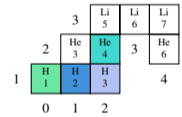
Z ↑

N →

N →

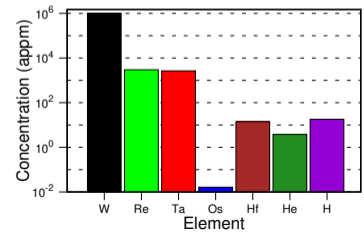


W



$\alpha$

14 MeV beam

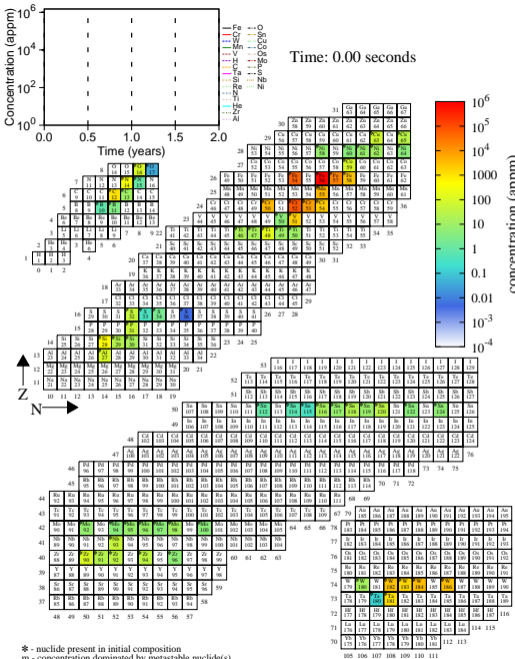


neutron

flux:  
10<sup>14</sup> particles  
cm<sup>-2</sup> s<sup>-1</sup>

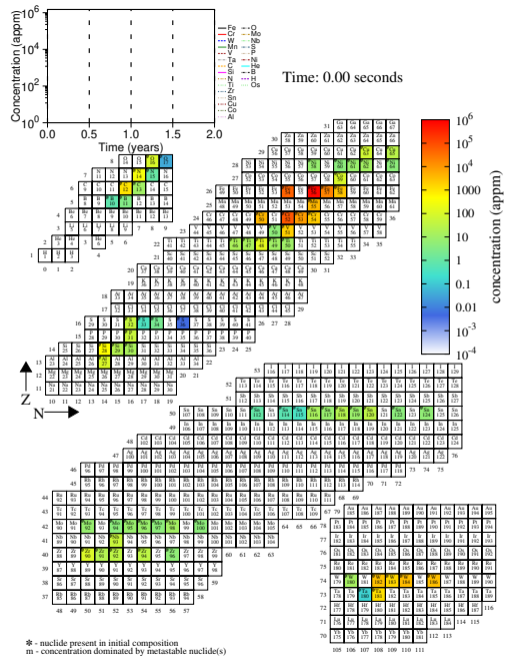
EUROFER at 8 MeV





Fusion  
FW  
←  
EUROFER

8 MeV  
Protons  
→  
 $10^{14}$   
flux



805 106 107 108 109 110 111

805 106 107 108 109 110 111

Fusion  
FW

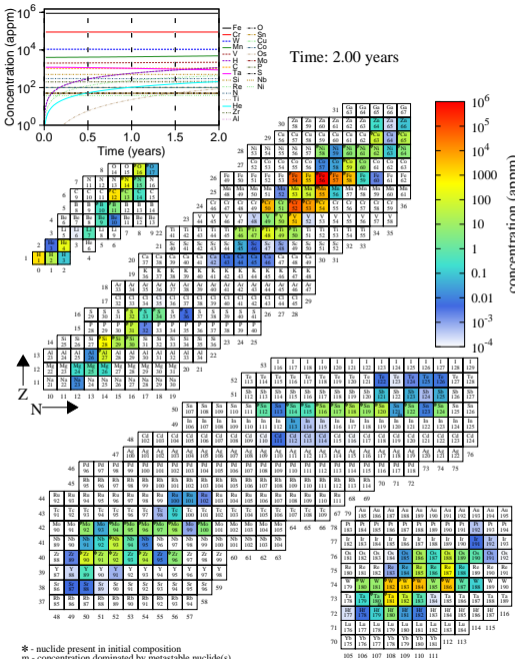


EUROFER

8 MeV  
Protons



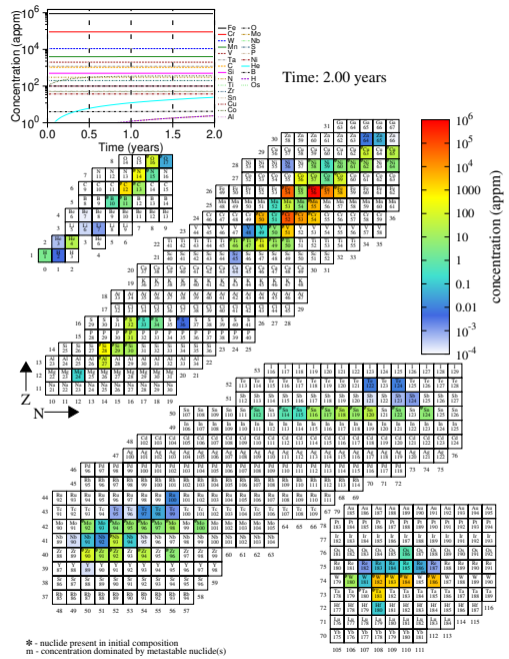
$10^{14}$   
flux



Fusion  
FW  
←

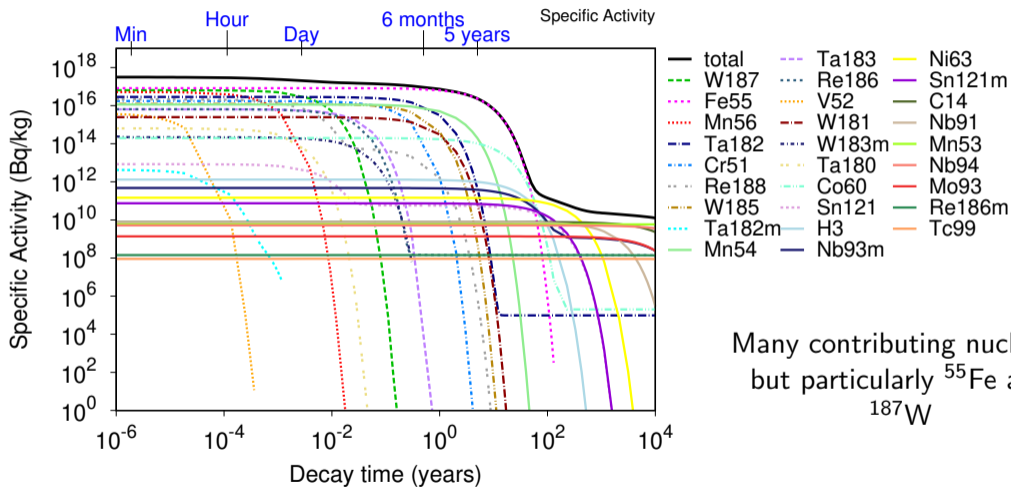
EUROFER

8 MeV  
Protons  
→  
10<sup>14</sup>  
flux



# Eurofer activation response – nuclide contributions

- Neutron fusion first wall (FW):

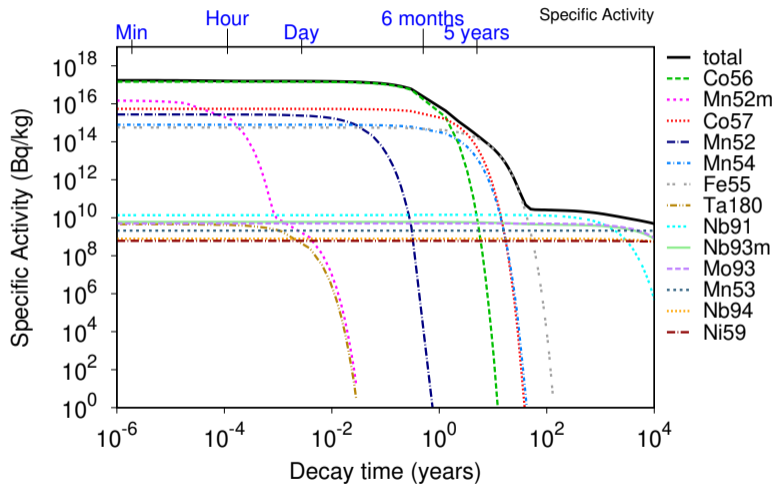


Many contributing nuclides,  
but particularly  $^{55}\text{Fe}$  and  
 $^{187}\text{W}$

- After 2-year irradiation

# Eurofer activation response – nuclide contributions

- Proton beam at 8 MeV:



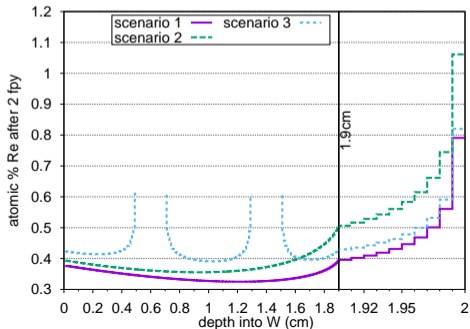
Much fewer nuclides produced ( $^{56}\text{Co}$ ,  $^{57}\text{Co}$ ,  $^{55}\text{Fe}$  are key) but radioactivity is comparable to neutron case initially

- After 2-year irradiation at  $10^{14}$  ions  $\text{cm}^{-2} \text{s}^{-1}$

## Caution

- FISPACT-II is zero dimensional – all atoms see same irradiation environment
- for neutrons, this is often a valid assumption due to long free paths of (fast) neutrons
  - ▶ but local variation in thermal flux can alter transmutation rates
- for charged particles, care should be taken to consider variation in energy spectrum with position (depth)
  - ▶ validity of monoenergetic assumption should be checked (e.g. with SRIM)

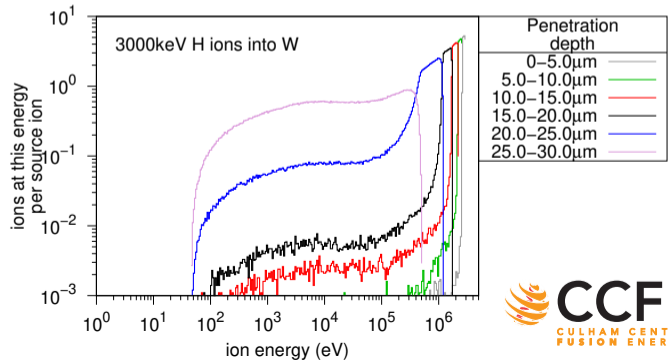
Combined transport and inventory simulations showing the local variation in Re production near to water pipes in W



## Caution

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SRIM calculation of 3 MeV protons into W shows that the ion spectrum changes with depth – only monoenergetic in first 10  $\mu\text{m}$



## Discussion

- FISPACT-II inventory simulations are a powerful tool for studying the impact that irradiation has on the chemical composition of materials
- with the latest nuclear data libraries, simulations are now routine for charged particle irradiation, including p, d, t,  $\alpha$ 
  - ▶ can provide nuclide finger-printing for use in planning the handling and transport of activated materials
  - ▶ comparing predictions for experimental conditions with simulations of fusion/fission environments can be useful in designing experiments that aim to understand the transmutation-induced impacts that materials will experience under operation
- Further reading:
  - ▶ Sublet, Eastwood, Morgan, Gilbert, Fleming, and Arter, "**FISPACT-II: An Advanced Simulation System for Activation, Transmutation and Material Modelling**", *Nucl. Data Sheets* **139** (2017) 77–137  
<http://dx.doi.org/10.1016/j.nds.2017.01.002>